

# Small Modular Nuclear Reactors

*~ The Canadian Context ~*

## A Slide Show

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Gordon Edwards, Ph.D., President,  
Canadian Coalition for Nuclear Responsibility  
e-mail: [ccnr@web.ca](mailto:ccnr@web.ca)

[www.ccnr.org](http://www.ccnr.org)

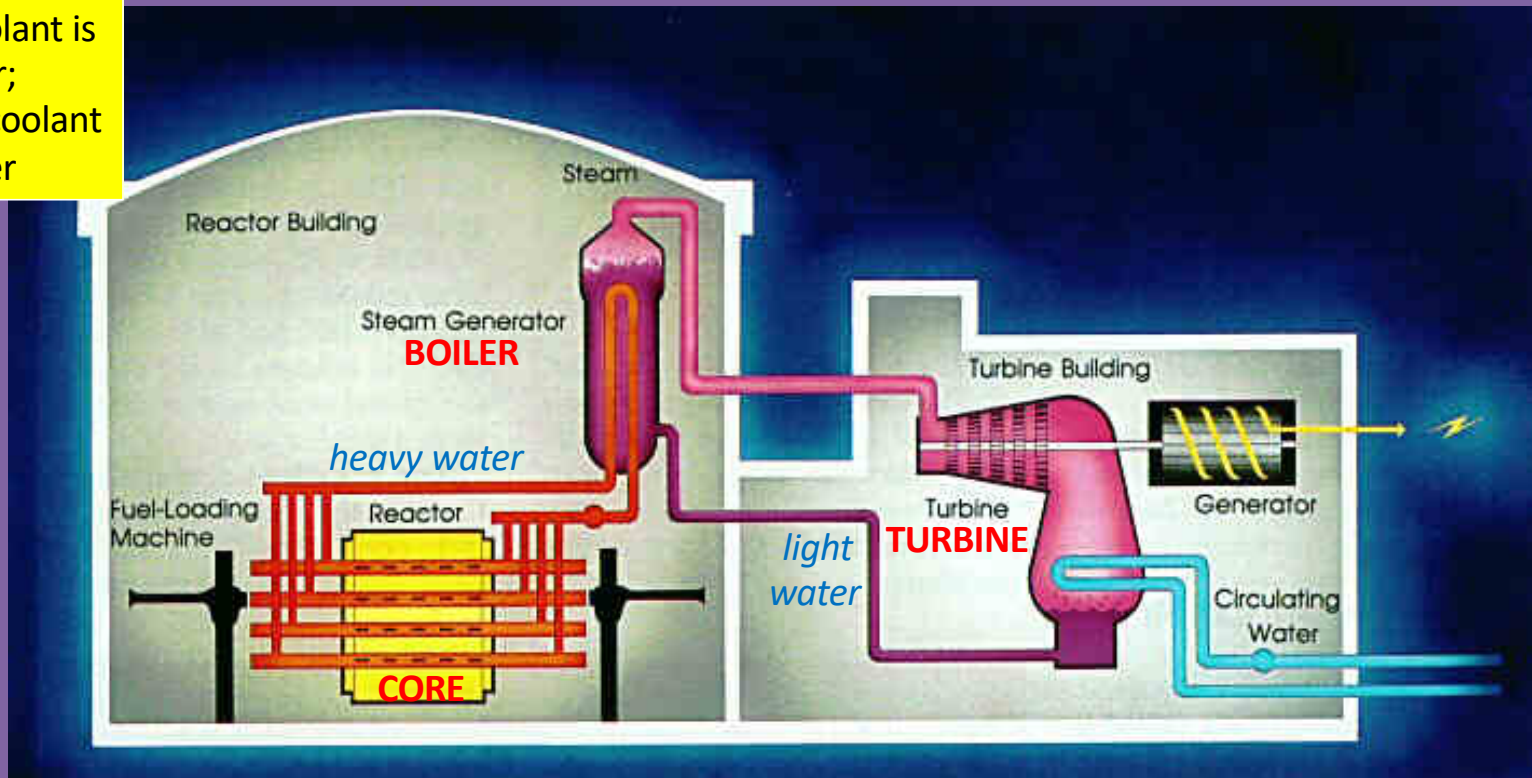
*The Point Lepreau CANDU  
Nuclear Reactor*



The Point Lepreau Reactor – a CANDU-6 model (about 600 MW electrical)  
The fuel is natural (unenriched) uranium; the coolant and moderator are heavy water.

## Inside a CANDU reactor

Primary coolant is heavy water; secondary coolant is light water



Uranium atoms are split in the CORE to heat water.  
Hot water is pumped to the BOILER (Steam Generator).  
Steam turns the blades of a TURBINE to generate electricity.  
*Everything in the "primary cooling circuit" (on the left) becomes RADIOACTIVE.*

## Small Modular Nuclear Reactors being “pre-evaluated” by Canadian Nuclear Safety Commission (CNSC)

VENDOR	SITE	SITE OWNER	BACKER(S)	REACTOR TYPE	DESIGN ACRO	MW	Fuel	Waste "Plan"
<a href="#">ARC Nuclear Canada Inc.</a>	Point Lepreau	NB Power	NB government, <a href="#">NB Power</a>	Sodium cooled fast reactor	ARC-100	100	13.1 % enriched uranium	“removed for recycling”
<a href="#">GE-Hitachi Nuclear Energy</a>	Darlington	OPG	<a href="#">OPG</a>	Boiling water reactor	BWRX-300e	300	3.4 to 4.95 % enriched uranium	‘segregated’ for “final disposal”
<a href="#">Moltex Energy</a>	Point Lepreau	NB Power	NB government, <a href="#">NB Power</a>	Molten Salt Reactor	SSR-W	300	Reactor grade plutonium	off site “disposal” of fuel waste
<a href="#">NuScale Power, LLC</a>	-	-	Bruce Power, <a href="#">OPG</a>	Pressurized water reactor	NuScale	60	< 4.95 % enriched uranium	wet then dry storage, long term in “national” plan
<a href="#">StarCore Nuclear</a>	Chalk River / Pinawa	AECL	<a href="#">CNL</a>	High-temperature gas cooled reactor	StarCore	10	15 % enriched uranium	off-site waste, entombment on site of reactor
<a href="#">Terrestrial Energy Inc.</a>	Darlington / Chalk River	OPG / AECL	<a href="#">OPG</a>	Molten Salt Reactor	IMSR	200	< 5 % enriched uranium	Liquid Fuel Waste
<a href="#">Ultra Safe Nuclear Corporation</a>	Chalk River	OPG	<a href="#">Global First Power (Proponent)</a> , <a href="#">CNL</a>	High-temperature gas cooled reactor	MMR-5 and 10	5-10	19.75 % enriched uranium	Spent fuel in “national” plan
<a href="#">X Energy, LLC</a>	Darlington	OPG	<a href="#">OPG</a>	High-temperature gas cooled reactor	Xe-100	80	15.5 % enriched uranium	Wet then dry storage, long term in “national” plan

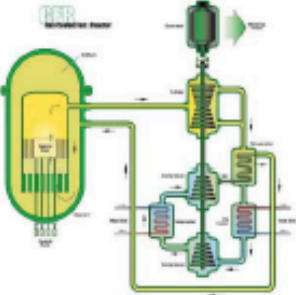
Some SMNRs are “fast” reactors, meaning the neutrons are not slowed down by a “moderator”. Without a moderator, the fissile material in the nuclear fuel must be much more concentrated.

Source:  
[www.stop-smrs.ca](http://www.stop-smrs.ca)

NOTE: This is an industry graphic from Moltex. TRISO fuel (for “slow” reactors) is described in the next slide

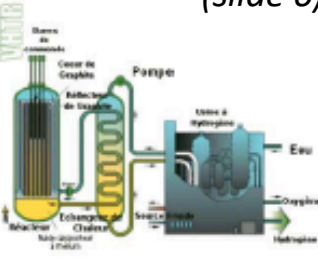
## What are the options for smaller nuclear?

**Gas cooled fast reactor**



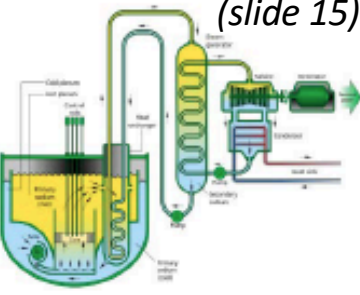
Costly high pressure system and expensive new solid fuel system

**TRISO fuelled reactors**  
*(slide 6)*



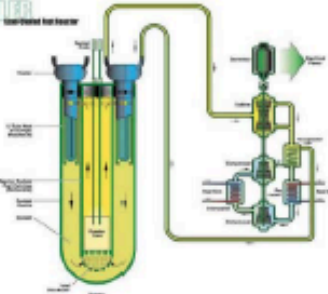
Excellent safety but published fuel cost estimates show cannot compete with gas/coal

**Sodium fast reactor**  
*(slide 15)*



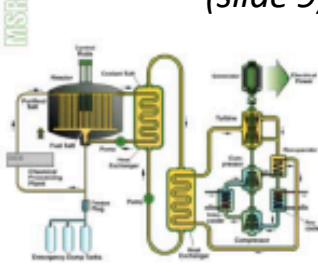
Highly dependent on engineered safety

**Lead cooled fast reactors**



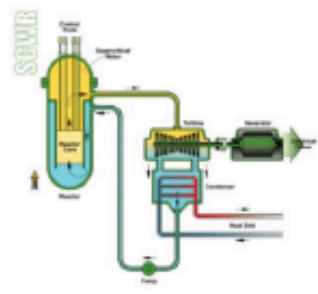
Better than sodium, but new materials needed

**Molten salt reactor**  
*(slide 9)*



MAYBE? Step change in intrinsic safety at lower cost than TRISO

**Supercritical water**

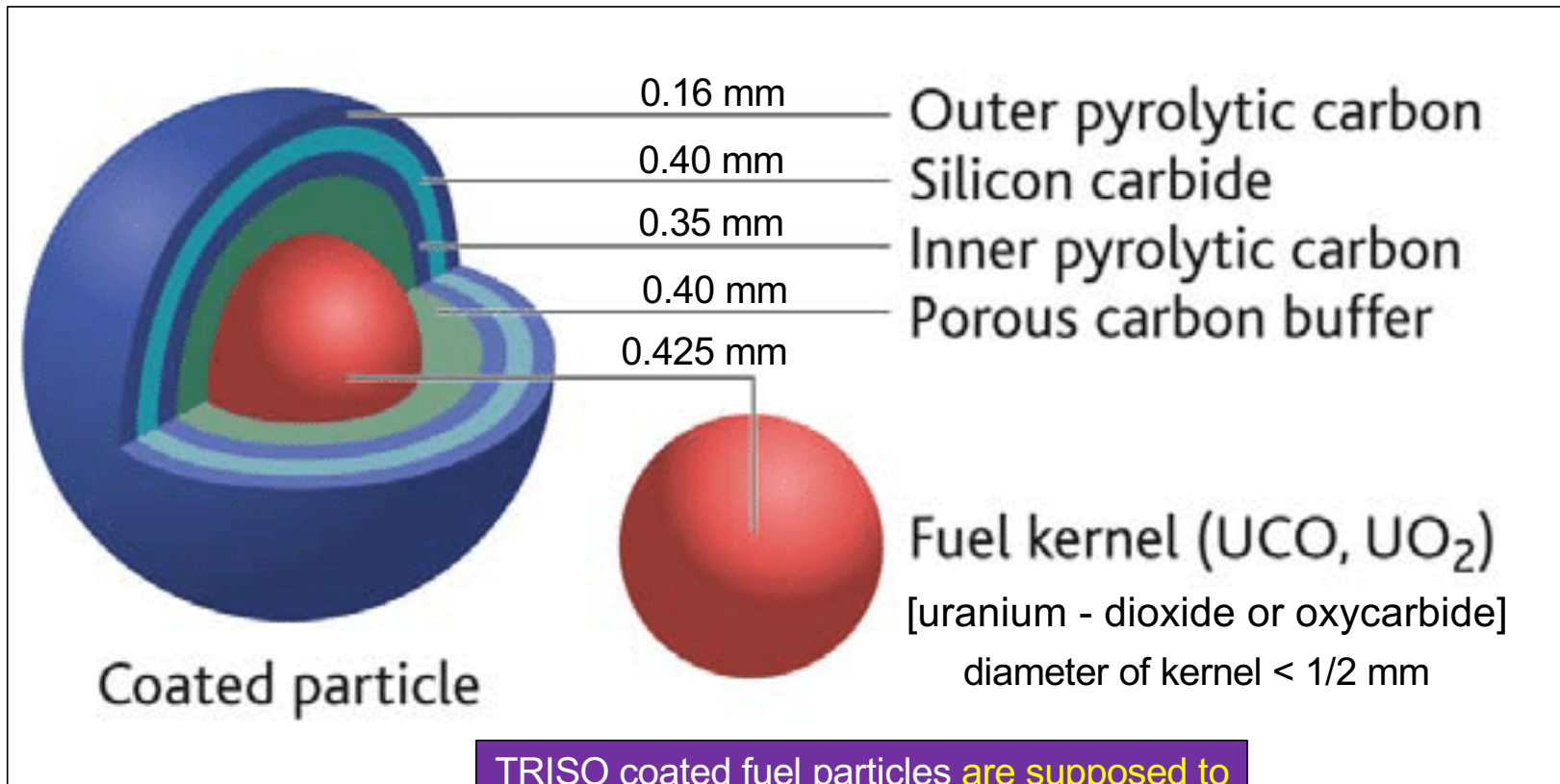


Not credibly cheaper

Most Small Modular Nuclear Reactors plan to use unorthodox fuels and unusual coolants.  
Fuel : Enriched Uranium (HALEU) or plutonium. Coolant: molten salt, liquid metal, or gas.

## WHAT IS TRISO FUEL?

Intended for any high temperature gas-cooled reactor that is “moderated” (i.e. uses “slow” neutrons)



TRISO coated fuel particles are supposed to contain all fission products including gases

## URANIUM ENRICHMENT and PLUTONIUM

In **natural uranium**, only 7 atoms out of 1000 are “fissile” (yielding energy). The other 993 atoms are non-fissile, but some “become” plutonium atoms. The earliest reactors, and all CANDU reactors, use natural uranium as fuel.

**Plutonium** is fissile and usable as a nuclear fuel, but it can only be extracted from used uranium fuel that has been MELTED or DISSOLVED into a LIQUID form.

Because **all reactor-produced plutonium is usable in nuclear weapons**, the “reprocessing” of used uranium fuel to extract plutonium is very controversial.

Most power reactors use **Low Enriched Uranium (LEU)** in which 30 to 50 atoms out of 1000 are fissile: NON-CANDU reactors use this kind of fuel.

Many proposed SMNRs are planning to use fuel that is **High Assay Low Enriched Uranium (HALEU)** in which 50 to 200 atoms out of 1000 are fissile.

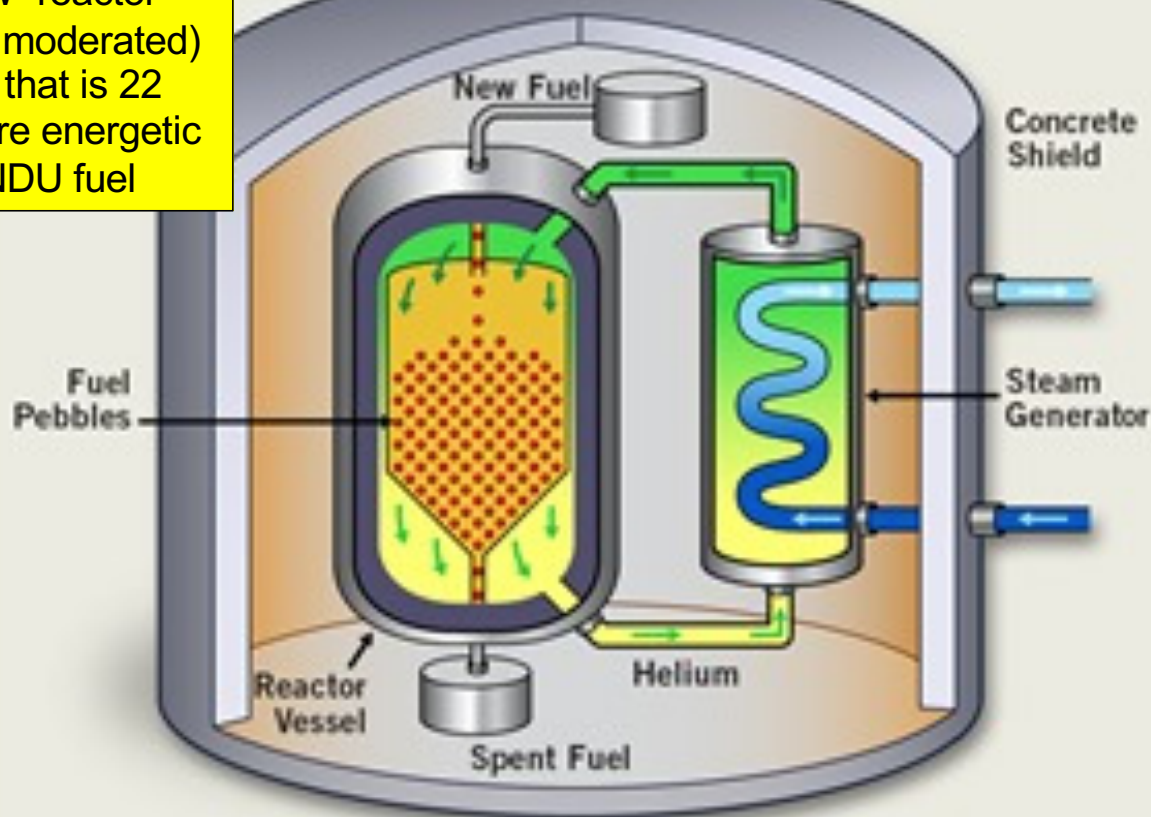
**Highly Enriched Uranium (HEU)**, with 200-1000 fissile atoms per 1000, is disallowed for commercial use and is also being phased out for research reactors – because **HEU is a powerful nuclear explosive material**.

*With higher enrichment, a **smaller mass of uranium fuel** is needed per unit of energy output. – and, for a given energy output, the used uranium fuel is **more radioactive per kilogram**.*



## The Xe-100 Pebble Bed Reactor

This “slow” reactor (graphite moderated) uses fuel that is 22 times more energetic than CANDU fuel



### NUCLEAR FUEL

Fuel is 15.5 percent enriched uranium

Each graphite “pebble” the size of a billiard ball is filled with thousands of tiny TRISO granules

Pebbles enter from above and exit below, each pellet is re-used up to three times.

### PRIMARY COOLANT

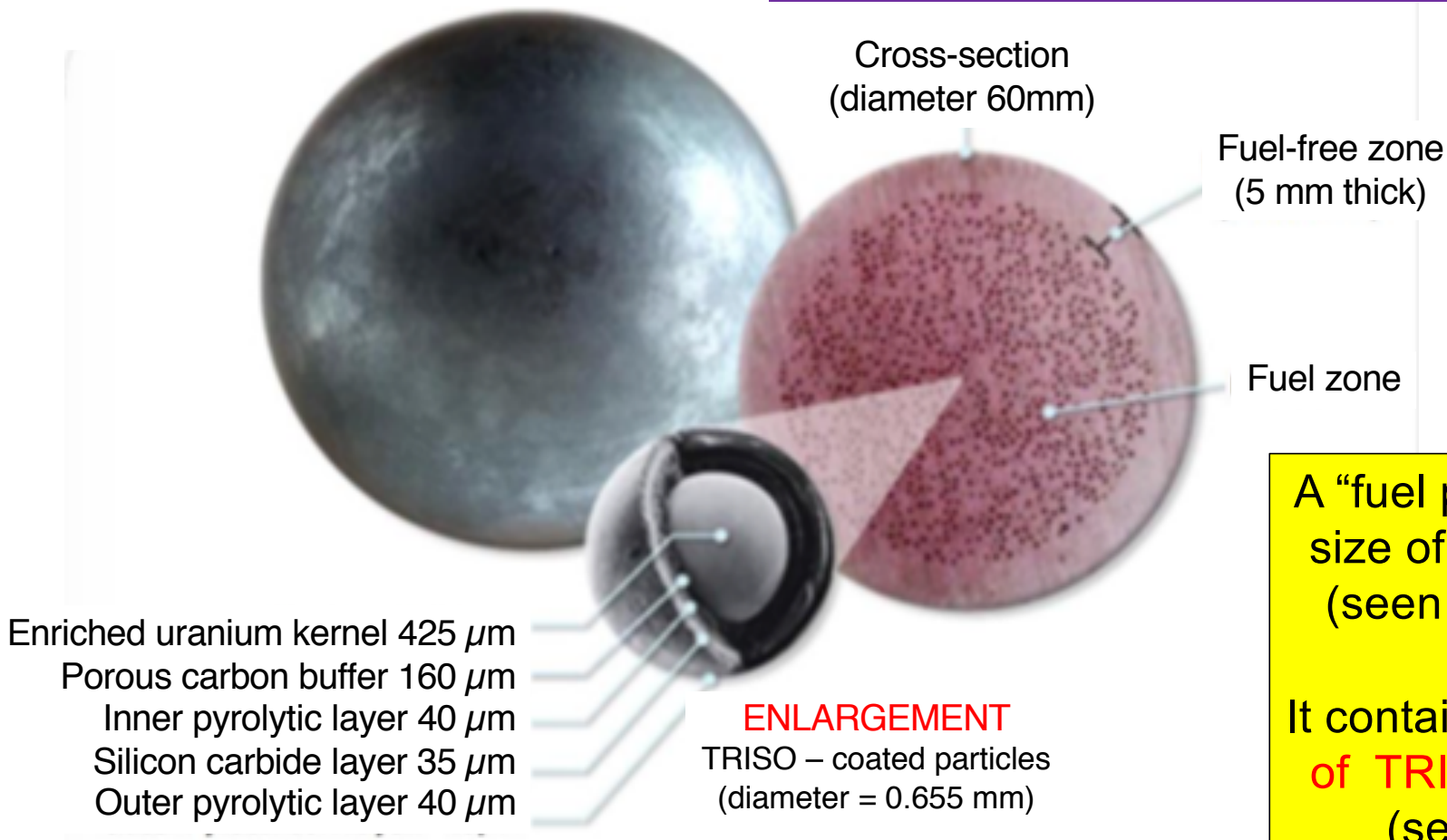
High temperature gas reaches temperature of close to 1000 deg C. – greater efficiency of electricity generation

Xe-100 Pebble Bed Reactor – helium-cooled – graphite moderated



# Fuel Pebble

## FUEL for The Xe-100 Pebble Bed Reactor

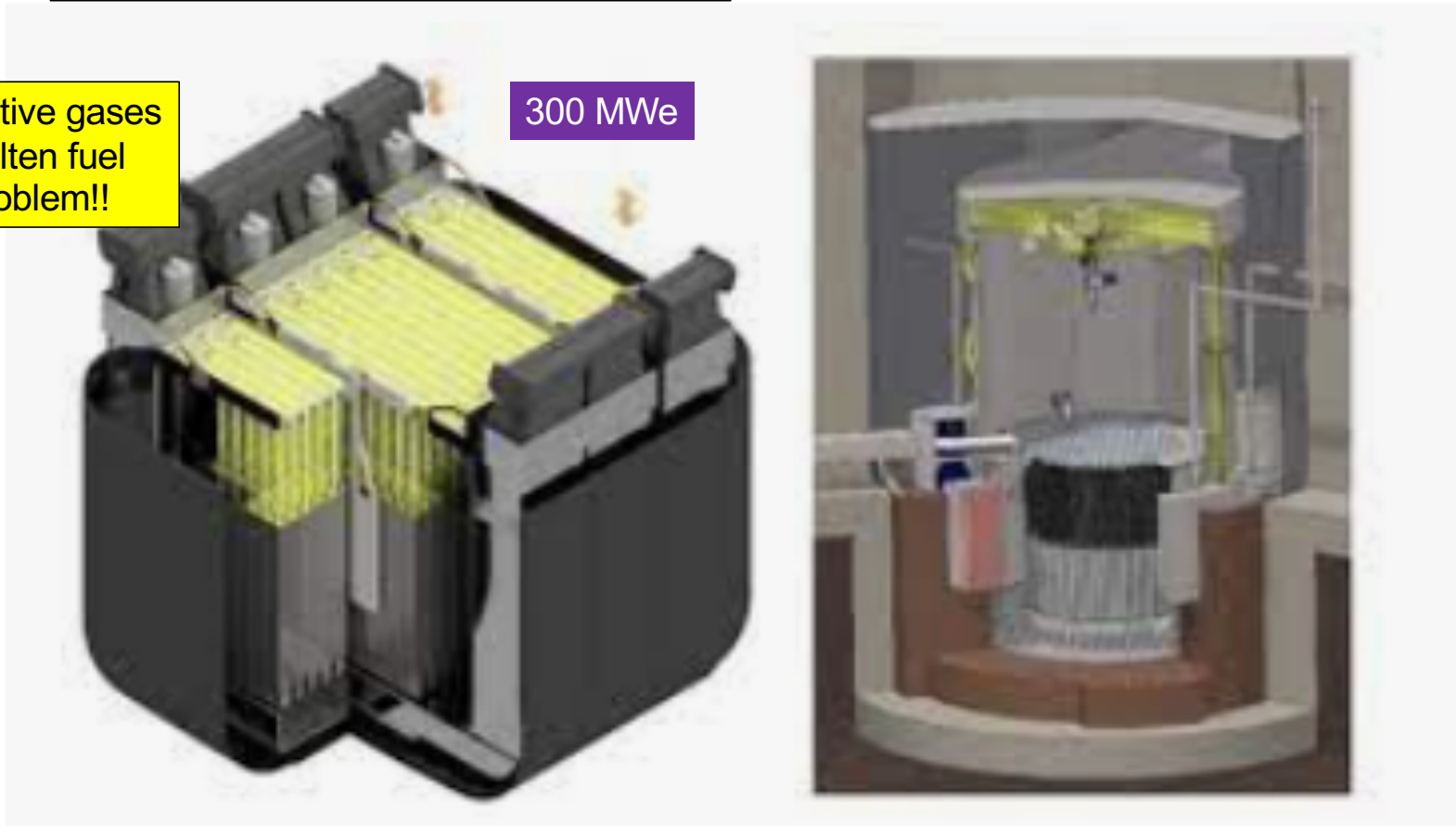


A “fuel pebble” is the size of a billiard ball (seen at your left).  
It contains **thousands of TRISO particles** (see slide 6)

The **Moltex Stable Salt Reactor – Wasteburner** (SSR-W) is a FAST REACTOR (no moderator)

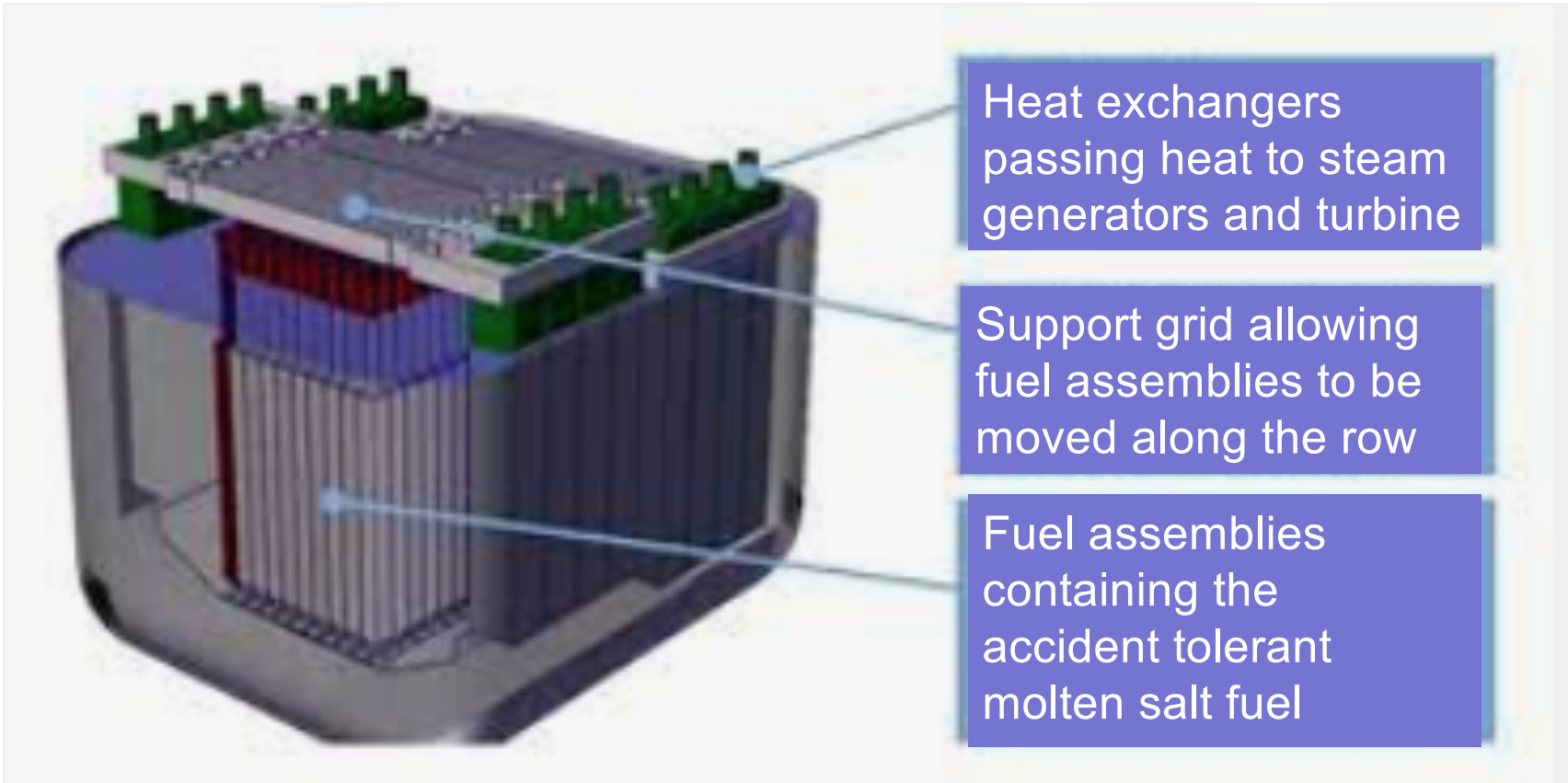
Radioactive gases from molten fuel are a problem!!

300 MWe



Each **SOLID** fuel rod is filled with a **LIQUID** solution of plutonium and transuranics in molten salt. The **PRIMARY COOLANT** is molten salt that carries heat to the **SECONDARY COOLANT** (water).

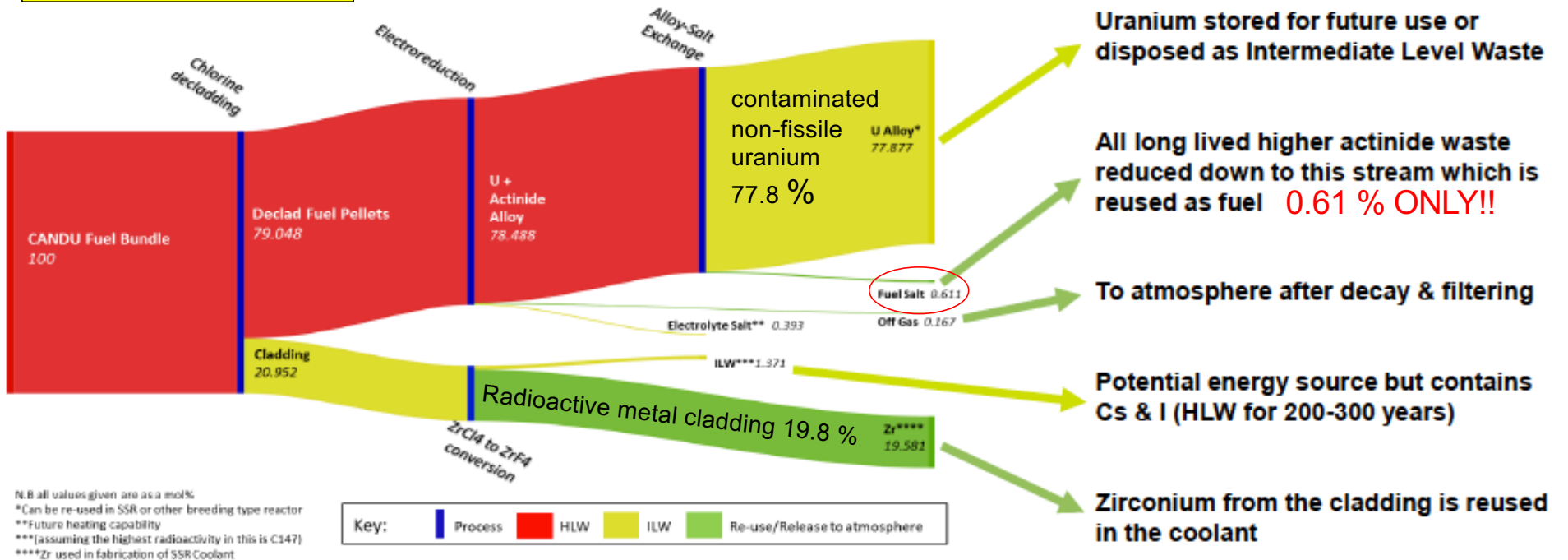
The Moltex Stable Salt Reactor – Wasteburner (SSR-W)



Moltex WATSS = Waste to Stable Salt (pyroprocessing unit) to extract plutonium mix from used CANDU fuel

**PYROPROCESSING**

To provide fuel for the Moltex SSR-W reactor

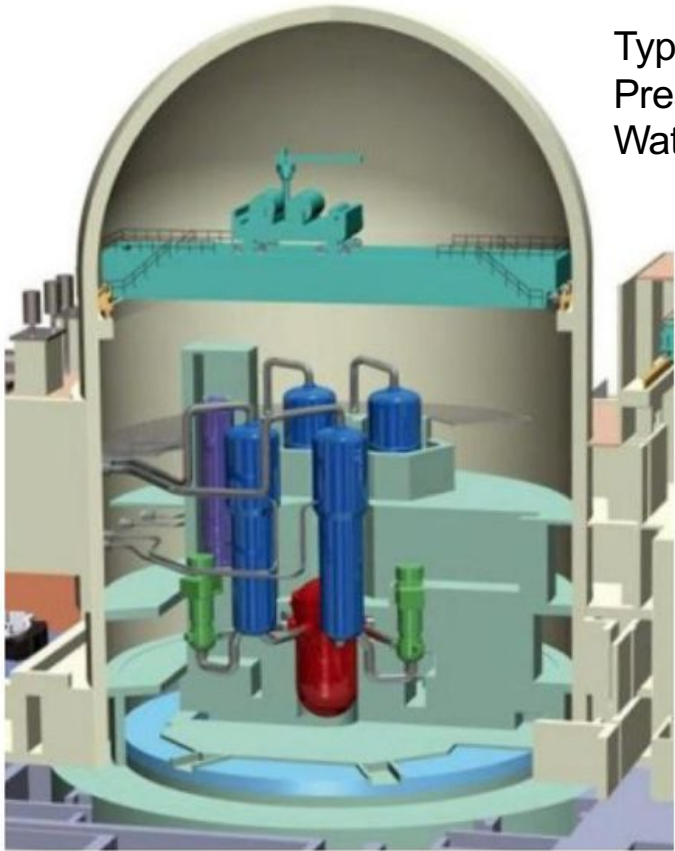


SSR Fuel = plutonium + other transuranics = 0.61 percent of the total mass of used CANDU fuel. This small fraction is used to fuel the SSR-W reactor. The other 99.39 percent is leftover waste.

# NuScale Pressurized Water Reactor

NuScale's combined containment vessel and reactor system

NUSCALE PWR  
60 megawatts  
= 1/10 of  
Point Lepreau



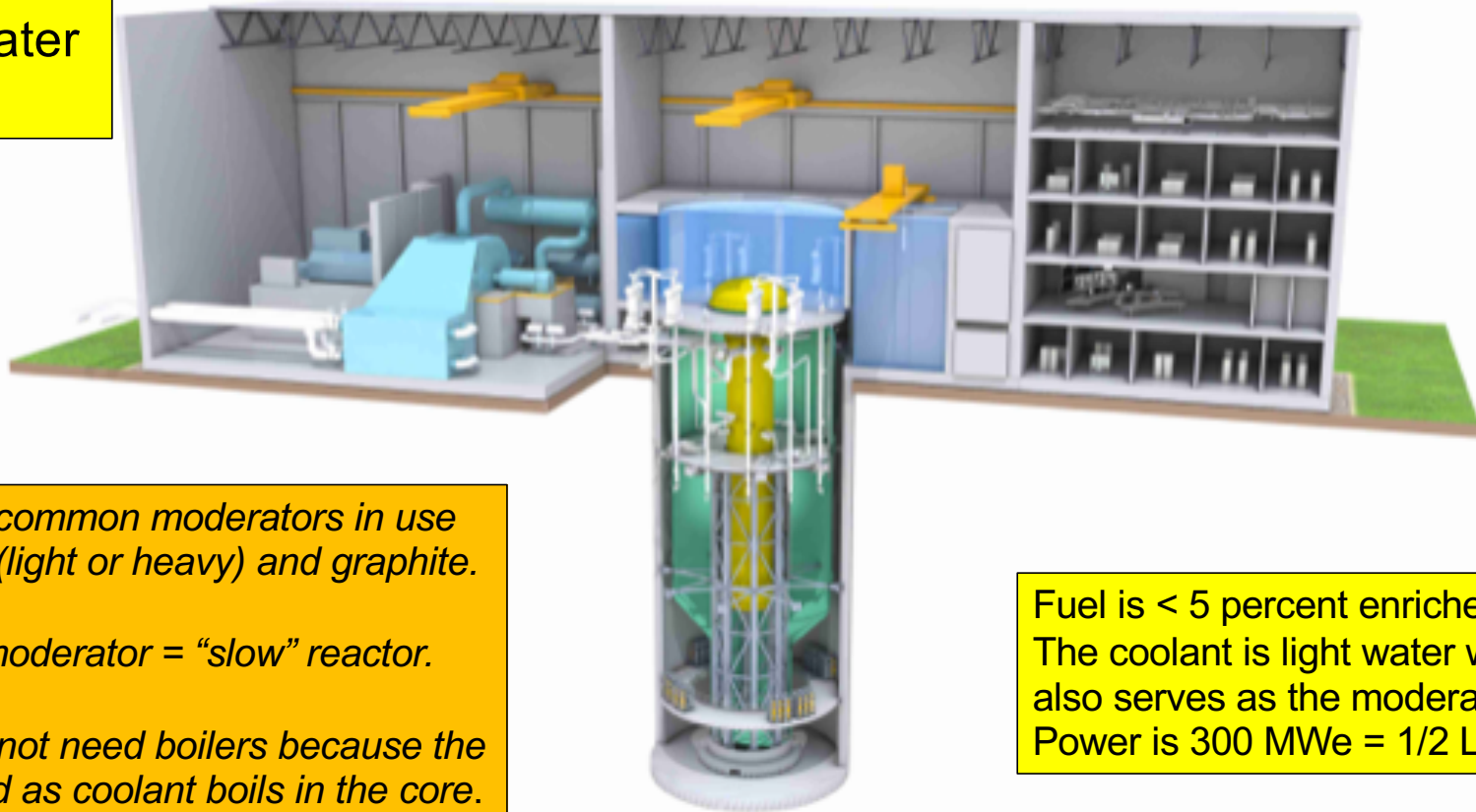
Typical PWR –  
Pressurized  
Water Reactor

\*Source: NRC

NuScale – pressurized light water reactor, scaled down conventional PWR

## BWRX-300 Boiling Water Reactor

BWRS-300  
Boiling Water  
Reactor



*The most common moderators in use are water (light or heavy) and graphite.*

*Use of a moderator = "slow" reactor.*

*BWRs do not need boilers because the water used as coolant boils in the core.*

Fuel is < 5 percent enriched.  
The coolant is light water which also serves as the moderator.  
Power is 300 MWe = 1/2 Lepreau

BWRX-300 = scaled down boiling water reactor – GE- Hitachi



MMR = Micro Modular Reactor (a “fission battery”)

Chalk River MMR  
15 MW of HEAT

(5 MW Electricity  
POSSIBLE)

This design is  
now undergoing  
an EA by CNSC

Helium gas-cooled  
TRISO fueled –  
secondary coolant  
Is molten salt for  
heat storage as  
backup power.

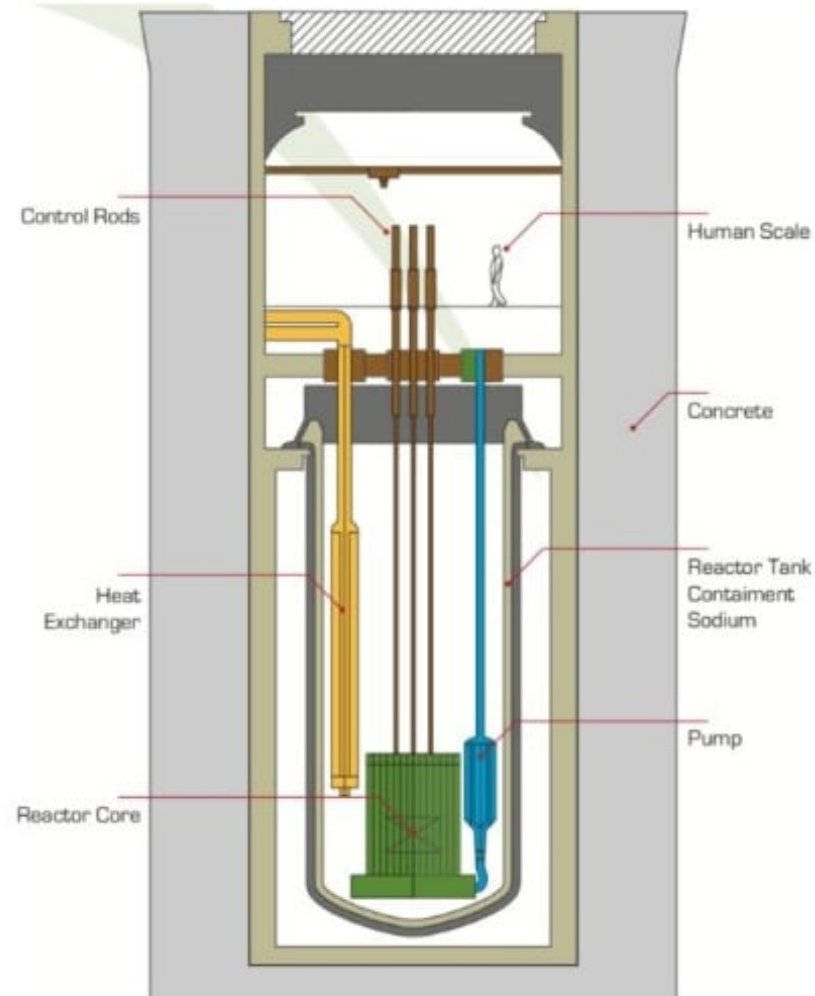
Reactor Pressure Vessel

Ultrasafe MMR = Micro Modular Reactor – 15 MWth – Helium-cooled – Molten Salt storage

ARC-100  
Liquid Metal Cooled  
Fast Breeder Reactor

Liquid sodium reacts  
violently with air and  
on contact with water.

## ARC-100 Reactor

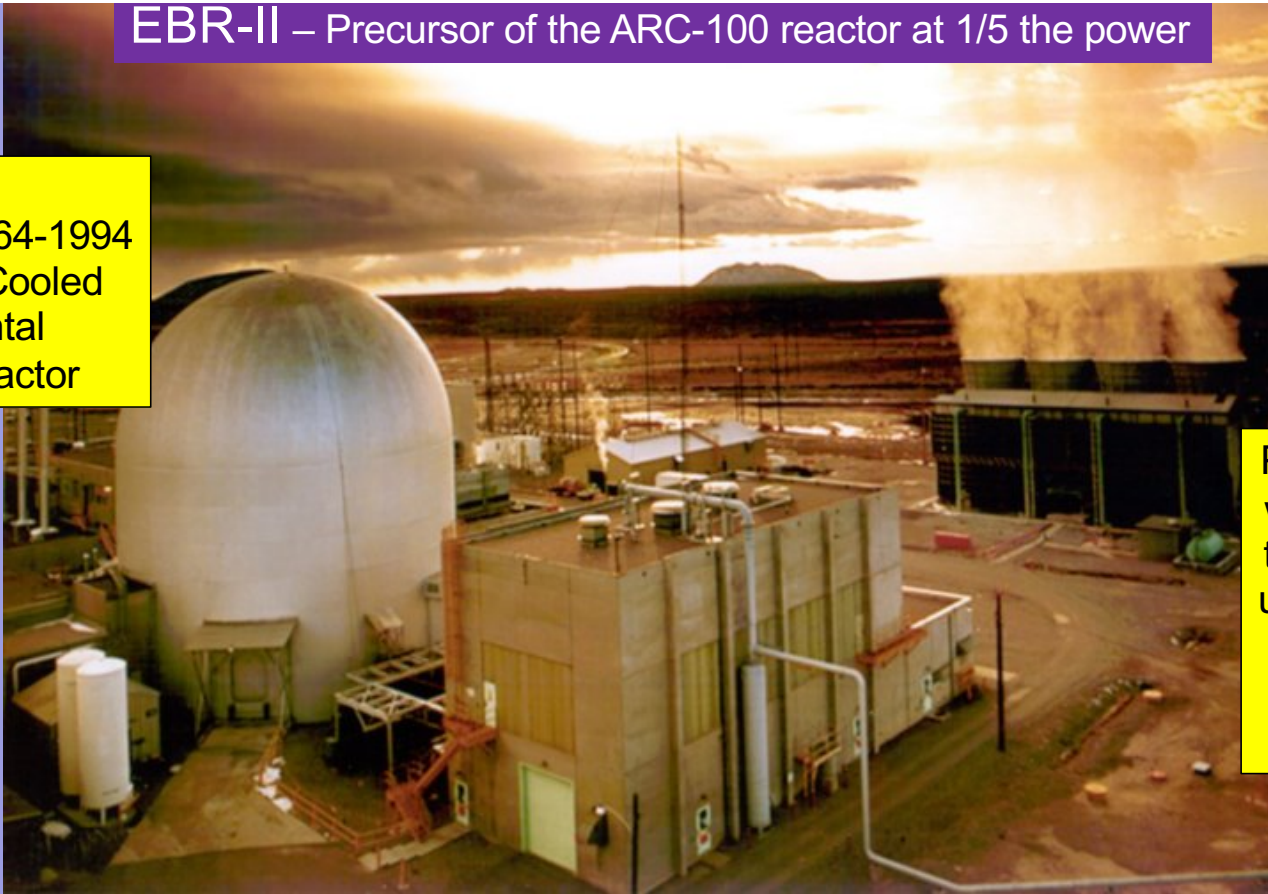


Enriched fuel  
(10.1, 12.1, 17.2 %)  
Is in METALLIC form  
(more compact)

*Larger reactors of this type  
were tried and abandoned in  
USA, UK, France, Germany*

EBR-II – Precursor of the ARC-100 reactor at 1/5 the power

EBR-II  
Fast reactor 1964-1994  
Liquid Metal Cooled  
Experimental  
Breeder Reactor

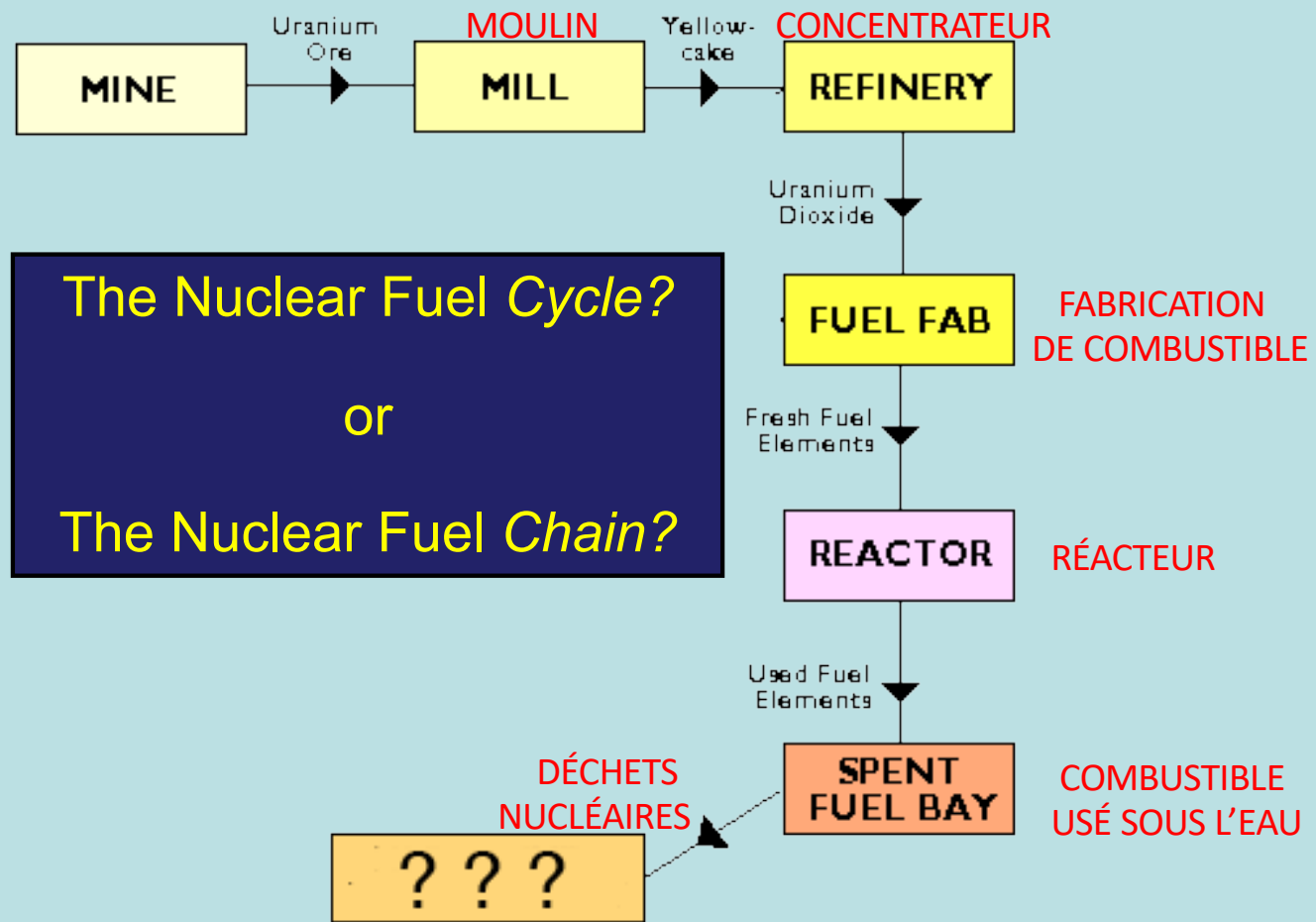


Pyroprocessing  
was developed  
to cope with the  
used EBR-II fuel

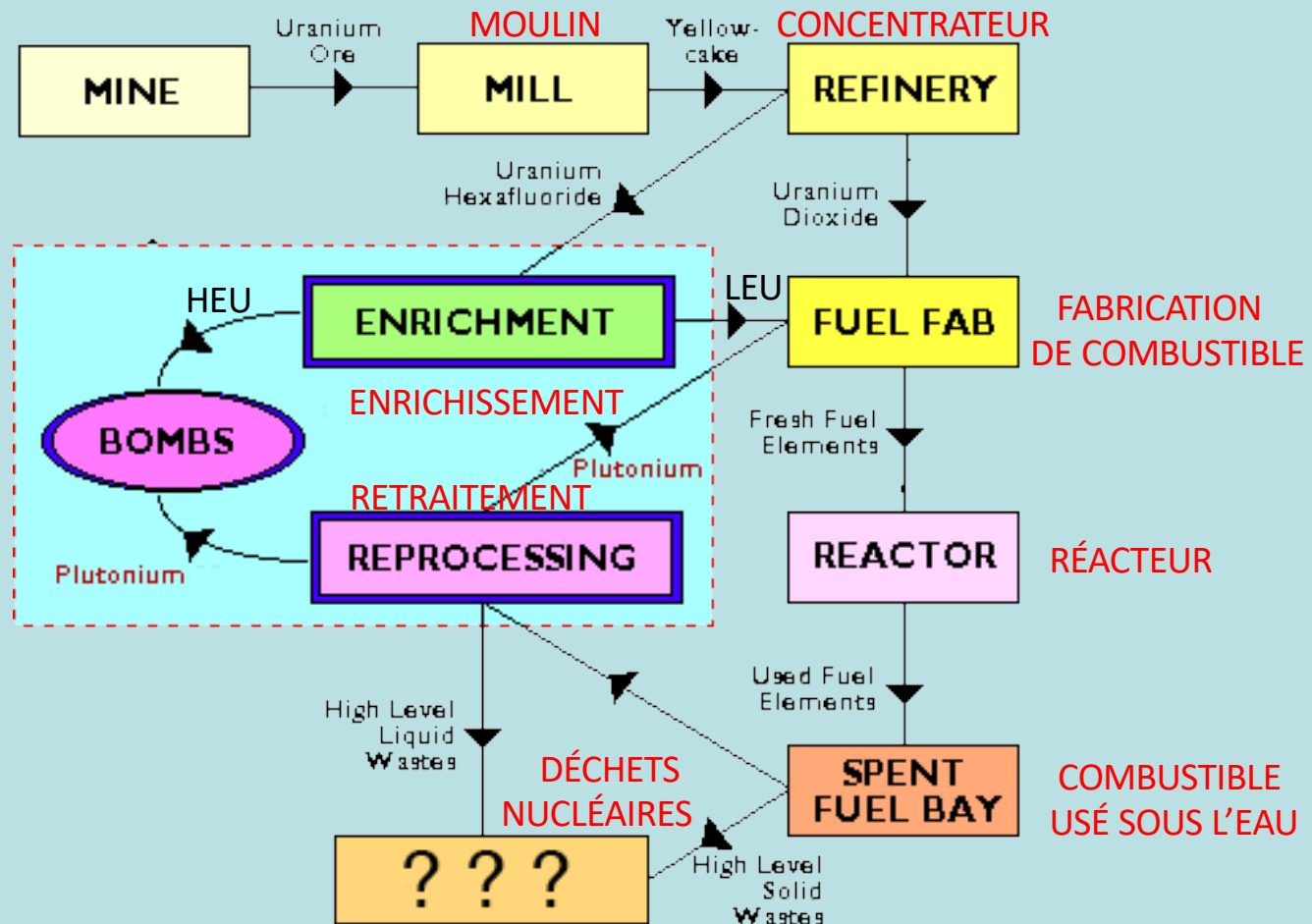
So far, not  
successfully!!

ARC-100 (100 megawatts) is supposedly modelled on EBR-II (20 megawatts)  
EBR-II fuel was 67 % enriched; ARC-100 fuel is 10.1, 12.1, 17.2 % enriched.  
A breeder reactor requires continuous reprocessing of the used nuclear fuel.

The most obvious thing about **the nuclear fuel cycle** is that it is NOT a cycle !!



. . . unless we decide to “recycle” the plutonium, opening Pandora’s box!!



## Reprocessing / Retraitement

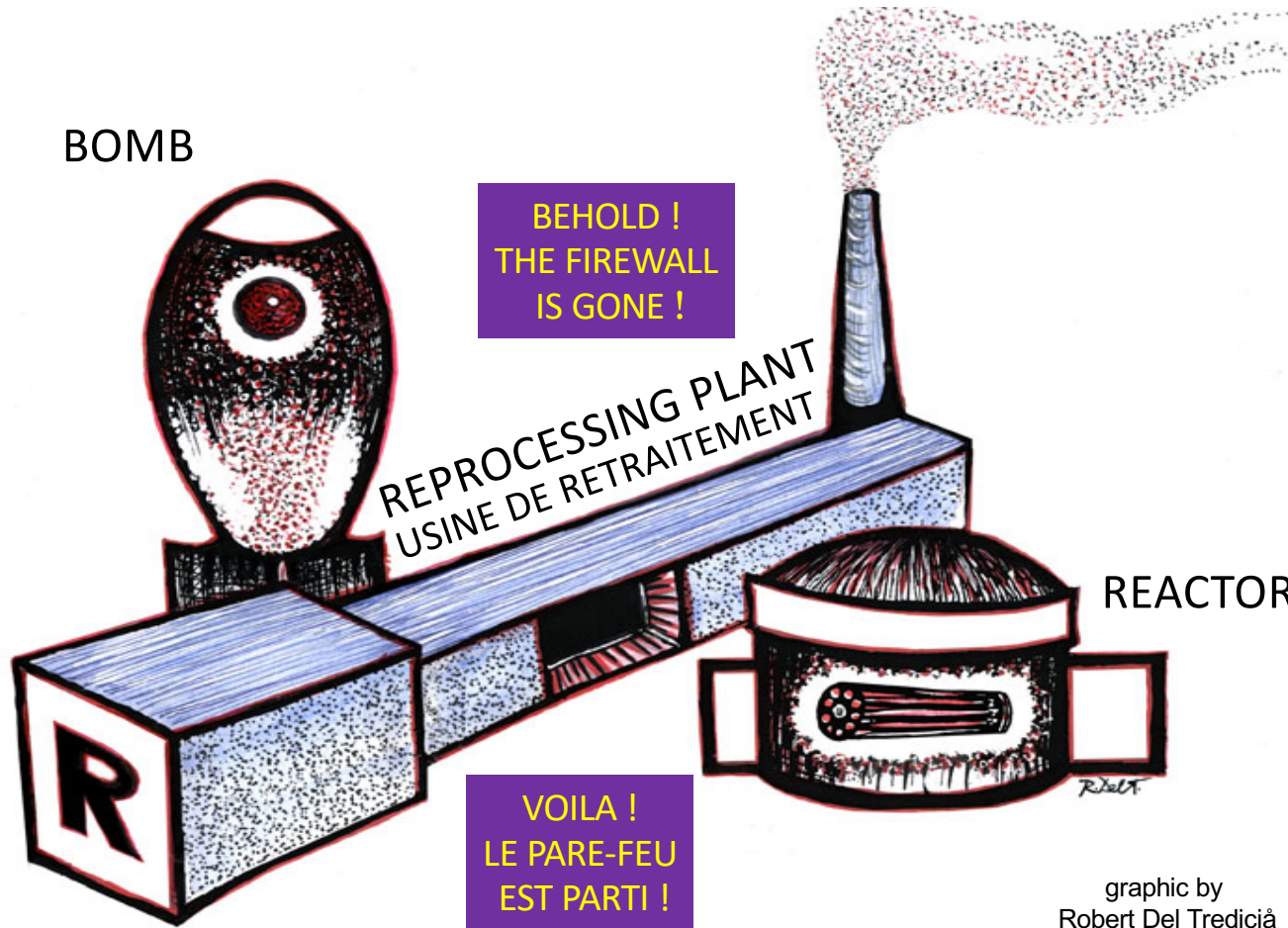


graphic by  
Robert Del Tredicià

**There is a radioactive firewall between used reactor fuel and bombs**

**Il existe un pare-feu radioactif entre le combustible usé des réacteurs et les bombes**





R = Recycling = Reprocessing = "Renaissance"

**A reprocessing plant removes the firewall and makes plutonium more accessible**

Une usine de retraitement supprime le pare-feu et rend le plutonium plus accessible

The End

[ccnr@web.ca](mailto:ccnr@web.ca)